

# Non-Resonant $n = 1$ Helical Core Induced by $m/n = 2/1$ Neoclassical Tearing Mode in JT-60U

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We have investigated the excitation mechanism of  $n = 1$  helical cores (HCs) with  $m/n = 2/1$  neoclassical tearing modes (NTMs) in JT-60U. It is found that the  $n = 1$  HC is observed with the mode frequency from several Hz to 6 kHz. This indicates that the resistive wall and the plasma control system do not induce  $n = 1$  HCs because the both time scales are different from the mode frequency. In addition,  $n = 1$  HCs appear to be the non-resonant mode from the two observations:  $n = 1$  HCs do not rotate with the plasma around the  $q = 1$  surface in the core and an  $n = 1$  HC is observed even when  $q_{\min.} > 1$ . It is also observed that the fluctuation due to an  $n = 1$  HC in the core region disappears with the stabilization of an  $m/n = 2/1$  NTM, implying that  $n = 1$  HCs are driven by  $m/n = 2/1$  NTMs. We revisit a quasi-linear MHD model where the  $n = 1$  HC is induced directly by the sideband of the current for the  $m/n = 2/1$  NTM, which potentially excites the non-resonant  $m/n = 1/1$  mode.

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## 1. Introduction

Helical cores (HCs) in tokamaks have been an important research topic for the fusion reactor because HCs can affect transports of impurities [1–4], energetic particles [5], or toroidal momentum [5]. In addition to the effect on transport by helical structures, recent studies suggested that magnetohydrodynamics (MHD) dynamo accompanied with HCs redistributes the current profile in the core and realizes sawtooth-free plasmas of hybrid scenarios as observed in DIII-D [6, 7], in which the minimum value of the safety factor profile,  $q_{\min.}$ , is kept slightly above unity by “flux pumping”.

Several theoretical models to explain the excitation mechanism of HCs have been proposed. The generation of the magnetic island by the excitation of resonant tearing modes (TMs) due to the decrease of the electron temperature around the  $q = 1$  surface was proposed [8], which is induced by hydrogen pellet injection. This model intended to explain “snake” structure in soft X-ray (SX) emission, which was observed in JET [9]. HC equilibria can be obtained with  $q_{\min.} \sim 1$  utilizing a three-dimensional MHD equilibrium solver [10, 11], which is consistent with experimentally observed HCs in plasmas having  $q_{\min.} \sim 1$  [4, 12–14]. The robustness of HC equilibria is confirmed by the linear and nonlinear MHD stability calculations [15]. If the beta value is sufficiently high, the

saturated  $m/n = 1/1$  quasi-interchange instability is excited and induce the dynamo loop voltage using 3D non-linear simulations [16]. On the other hand, in recent experimental studies, HCs are induced by  $n = 1$  magnetic perturbation with the external coils and characteristics of HCs are investigated with taking account of reconstructed three-dimensional MHD equilibria in DIII-D [7, 17, 18]. The EAST team recently showed that the two-dimensional mode structure of a long-lived helical mode by combined use of tomography and a singular value decomposition, showing an  $m/n = 1/1$  mode structure and a substantial  $m/n = 2/2$  component where  $q_{\min.}$  is kept slightly above unity [19].

Though many experimental and theoretical studies have been reported as mentioned above, the study on the excitation mechanism of HCs with MHD modes except for  $m/n = 1/1$  modes has not been reported as far as we know. Recent our study [20] showed that a “coupling” on the phase of the MHD mode is observed between  $n = 1$  HCs and  $m/n = 2/1$  TMs: (1) The  $m/n = 2/1$  TM and the  $n = 1$  HC rotate keeping  $f_{m/n=1/1(\text{HC})} = 2f_{m/n=2/1(\text{TM})}$ , resulting in the phase locking. Here,  $f_{m/n=1/1(\text{HC})}$  and  $f_{m/n=2/1(\text{TM})}$  are the rotation frequency of the  $n = 1$  HC and the  $m/n = 2/1$  TM in the poloidal direction, respectively. (2) The core is shifted to the high-field side when the O-points of the  $m/n = 2/1$  magnetic island line up in the midplane as shown in Fig. 1. In this study, the “coupling” means cou-

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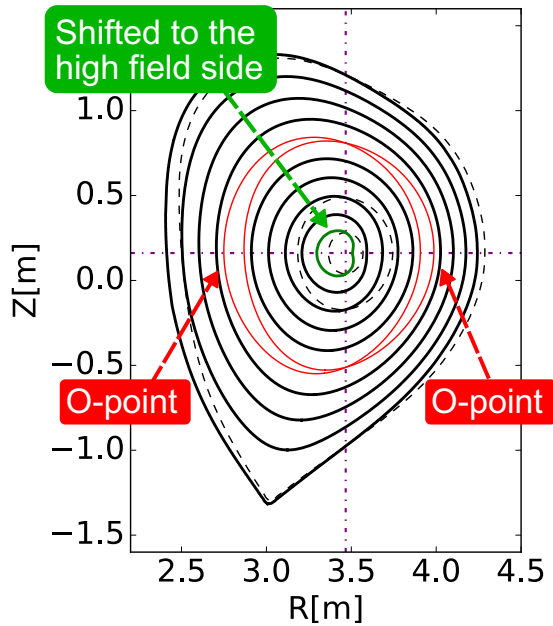


Fig. 1 Schematic view of the poloidal cross sections showing the phases of the  $m/n = 2/1$  TM and the  $m = 1$  HC based on the view of the coupling described in the previous study [20]. The core is shifted to the high-field side when the O-points of the  $m/n = 2/1$  magnetic island line up in the midplane. To keep this configuration, the  $m/n = 2/1$  TM and the  $n = 1$  HC have to rotate satisfying  $f_{m/n=1/1(\text{HC})} = 2f_{m/n=2/1(\text{TM})}$ . The red curves represent the  $m = 2$  magnetic island. The black and green curves show the flux surfaces with  $m = 1$  deformation. The black dashed lines show the flux surfaces at  $\rho \sim 0.1, 0.3$ , and  $1.0$  without  $m = 1$  deformation. The cross points of the purple dashed lines show the magnetic axis without  $m = 1$  deformation. The deformed flux surfaces are obtained by adding artificial  $m/n = 1/1$  deformation and artificial  $m/n = 2/1$  magnetic islands to the axisymmetric equilibrium.

pling satisfying the two characteristics (1) ~ (2) mentioned in this paragraph. In our previous study [20], though possible excitation mechanisms of HCs were proposed such as the effect of the resistive wall where the resistive wall time  $\tau_w \sim 10$  ms [21] in JT-60U or helical equilibria with  $q_{\min.} \sim 1$ , these candidates were not investigated. This is mainly because the coupling was only studied with TMs having low mode frequencies ( $\leq 20$  Hz), which is mentioned as Low Frequency Mode (LFM) in the previous study [20]. In addition, it is also required to investigate whether the magnetic fluctuation induced by the plasma control system, where the time interval of the control of the plasma position is  $250 \mu\text{s}$  (4 kHz), can induce HCs or not. This is because HCs induced by the external  $n = 1$  component of the magnetic fluctuation are observed in DIII-D and ASDEX [4]. Though the dominant component of the magnetic fluctuation induced by the plasma control system is the  $n = 0$  component, the  $n = 1$  component can be induced due to the misalignment of the coils in JT-60U.

In this paper, we report investigations of the excitation mechanism of  $n = 1$  HCs in JT-60U. It is found that the mode frequency with the coupling can be 6 kHz, which indicates that the resistive wall and the plasma control system do not induce the coupling. In addition,  $n = 1$  HCs appear to be the non-resonant mode from two observations. First, it is found that  $n = 1$  HCs do not rotate with the plasma around the  $q = 1$  surface in the core region. Second, the coupling is observed even when  $q_{\min.} > 1$ . It is also observed that the temperature fluctuation due to an  $n = 1$  HC in the core region disappears with the stabilization of an  $m/n = 2/1$  neoclassical tearing mode (NTM), implying that  $n = 1$  HCs are driven by  $m/n = 2/1$  NTMs. This perspective,  $n = 1$  HCs are driven by  $m/n = 2/1$  NTMs, is supported by the observation that the saturated amplitude of the  $m/n = 1/1$  component of the radial displacement is smaller than that of the  $m/n = 2/1$  component. We revisit a quasi-linear MHD model where the  $n = 1$  HC is induced directly by the sideband of the current for the  $m/n = 2/1$  NTM, which potentially excites the non-resonant  $m/n = 1/1$  mode. The model consistently describes the characteristic of the coupling,  $f_{m/n=1/1(\text{HC})} = 2f_{m/n=2/1(\text{TM})}$ .

This article is organized as follows: in Sec. 2, the heating system and diagnostic of JT-60U are introduced. In Sec. 3, the investigations of  $n = 1$  HCs with  $m/n = 2/1$  NTMs having the mode frequencies of several kHz are introduced in Sec. 3.1 first, then, the observation of an  $n = 1$  HC, where  $q_{\min.}$  is greater than unity, is introduced in Sec. 3.2. In these subsections, the observed mode structures are compared with the characteristics of the coupling mentioned in this section. The causality between the  $n = 1$  HC and the  $m/n = 2/1$  NTM is investigated in Sec. 4. The discussion focusing on the excitation mechanism is given in Sec. 5. We revisit the quasi-linear MHD model where the  $m/n = 1/1$  mode is induced by the sideband. Then the study is summarized in Sec. 6.

## 2. Heating System and Diagnostic of JT-60U

JT-60U is a tokamak device where typical major radius is 3.4 m and typical minor radius is 0.9 m [22]. The toroidal direction of the toroidal magnetic field and the plasma current is the clockwise direction in the analyzed discharges. In JT-60U, plasmas are heated by eleven neutral beam injectors (NBIs) with positive-ion source (P-NBI), whose acceleration energies are about 85 keV. Two (#9 and #10) of the tangential P-NBIs are co-current direction (co-direction) and other two (#7 and #8) tangential P-NBIs are counter-current direction (ctr-direction). Here, the co-direction and the ctr-direction are defined as the same direction and the opposite direction to the plasma current, respectively. The other seven NBIs inject neutral beams to the plasma perpendicularly (PERP P-NBI). The electron cyclotron heating (ECH) with gyrotron is also available. The ECH is used to stabilize NTMs by elec-

tron cyclotron current drive (ECCD) [23] as well as to heat plasmas.

Figure 2 shows the sight lines of SX measurement (Upper SX) and the measured plane by electron cyclotron emission (ECE) measurement by heterodyne radiometers [24] and the polychromator [25], which are used to study the radial mode structure. The MHD equilibrium in Fig. 2 is obtained at 9.5 s of 49713 (Figs. 3, 4, and 5). The difference in the toroidal angle of the locations of SX measurement and ECE measurement are about 160 degrees. The ion temperature and the toroidal velocity are measured by charge exchange recombination spectroscopy (CXRS) [26], which is the same as CXRS1 in [20]. The toroidal and poloidal mode numbers are estimated with magnetic probes arranged toroidally and poloidally. The profile of the safety factor ( $q$ ) is calculated with 16 channels of the motional Stark effect (MSE) measurement [27, 28] on the P17 port of JT-60U, which measures the polarization angles from the core region to the edge region of plasmas. The  $q$ -profile is reconstructed with the MEUDAS code [29] to reconstruct MHD equilibria assuming the toroidally axisymmetric geometry by solving the Grad-Shafranov equation. The MHD equilibrium is reconstructed so that the  $q$ -profile from the MHD equilibrium is consistent with the  $q$ -profile from the polarization angle of MSE measurement in the P17 port. The error of the estimated  $q$ -profile comes from the error of the measured polarization angle in MSE measurement. In JT-60U, the error of the polarization angle is about 0.1 degrees. The error of the estimated  $q$ -profile will be discussed in the case of Fig. 6 (e). The reconstructed MHD equilibrium can be

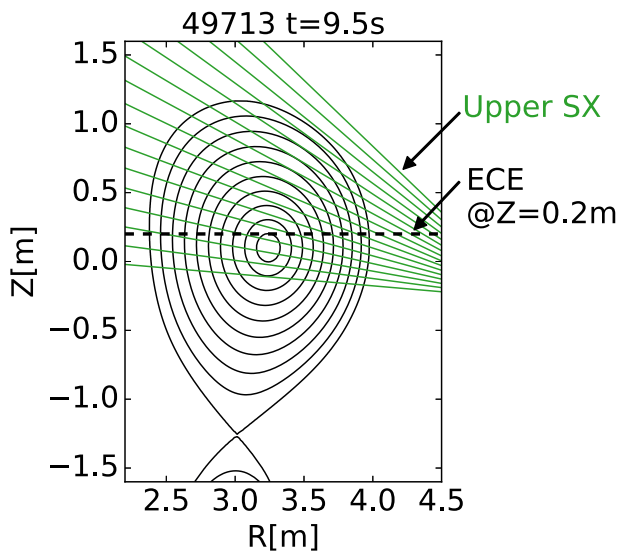


Fig. 2 The sight lines of SX measurement and the measured plane by ECE measurement. The difference in the toroidal angles between the locations of SX measurement and ECE measurement are about 160 degrees. The MHD equilibrium is obtained at 9.5 s of 49713.

obtained every 0.01 s, which is determined by the time resolution of the polarization angle from MSE measurement.

### 3. Experimental Condition of Observation of $n = 1$ HC with $m/n = 2/1$ NTM

#### 3.1 $n = 1$ HC with $m/n = 2/1$ NTM whose frequency is several kHz

$n = 1$  HCs with  $m/n = 2/1$  NTMs, whose observed frequencies are several kHz, are investigated in the stabilization experiments of  $m/n = 2/1$  NTMs by ECCD [23]. The plasma parameters in the analyzed plasmas are shown in Table 1. In these discharges, NTMs are excited by the high injection power ( $\sim 20$  MW) of NBI and then the mode frequency of NTMs is increased with the increased toroidal rotation velocity by the tangential NBIs. As results, the mode frequencies of NTMs become several kHz. Due to the large net toroidal torque to the ctr-direction, the rotation directions of NTMs are the ctr-direction toroidally and the electron diamagnetic direction poloidally in discharges of Table 1 when the mode frequencies of  $m/n = 2/1$  NTMs are several kHz.

In the analyzed discharges, the coherent fluctuations

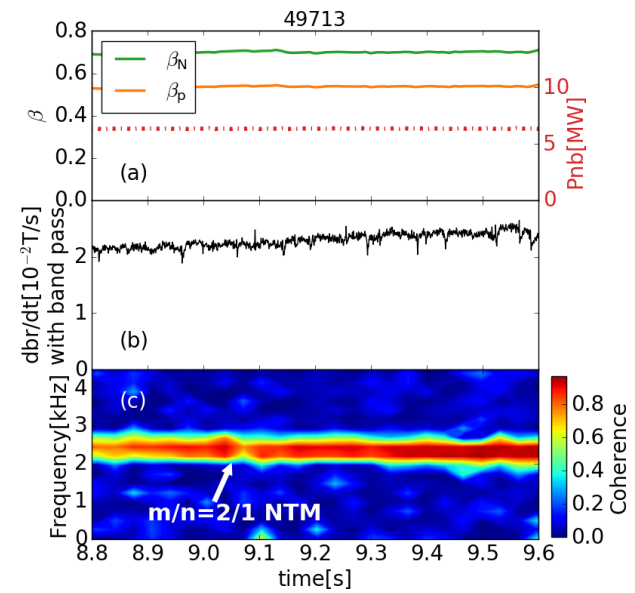


Fig. 3 Time evolution of the signals from an experiment of the stabilization of  $m/n = 2/1$  NTMs by ECCD in JT-60U: (a) the poloidal beta  $\beta_p$ , the normalized beta  $\beta_N$ , and the total injection power of NBIs (dotted-dashed red line), (b) the radial component of the magnetic fluctuation due to the  $m/n = 2/1$  NTM obtained with the band pass filter (from 1.8 kHz to 3.2 kHz), and (c) the spectrum of coherence of SX signals from a slight line viewing the core and the magnetic fluctuation. In (a),  $\beta_N$  is about 0.7 and is much lower than the no-wall beta limit  $\beta_{N(\text{no-wall})} \sim 2.93$  estimated with  $3li$  [32]. Here,  $li$  is the internal inductance.

Table 1 The plasma parameters of 39 discharges where  $m/n = 2/1$  NTMs with  $n = 1$  HCs, whose mode frequencies are several kHz, are observed. The analyzed discharges are made after the installation of ferrite steel tiles on the vacuum vessel to reduce the toroidal magnetic field ripple [30].

|   |  |
|---|--|
| $\beta_N$   | 0.6 ~ 0.8 % m T MA <sup>-1</sup>                 |
| $\beta_p$   | 0.46 ~ 0.68                                      |
| Total neutron emission rate                         | $1.1 \times 10^{15} \sim 2.5 \times 10^{15}$ n/s |
| Plasma current                                      | 1.5 MA   |
| Magnitude of magnetic field at magnetic axis        | 3.7 T  |
| Major radius  | 3.18 m   |
| Minor radius  | 0.8 m  |
| Plasma volume                                       | 55 ~ 56 m <sup>3</sup>                           |
| Radial position of axis                             | 3.23 ~ 3.25 m                                    |
| Vertical position of axis                           | 0.10 ~ 0.11 m                                    |
| Plasma triangularity                                | 0.18 ~ 0.20                                      |
| Plasma elongation                                   | 1.4 ~ 1.5  |
| $q_{95}$ (the safety factor on 95% of flux surface) | 4.05 ~ 4.09                                      |
| Plasma-wall separation in low field side            | 0.58 ~ 0.60 m                                    |

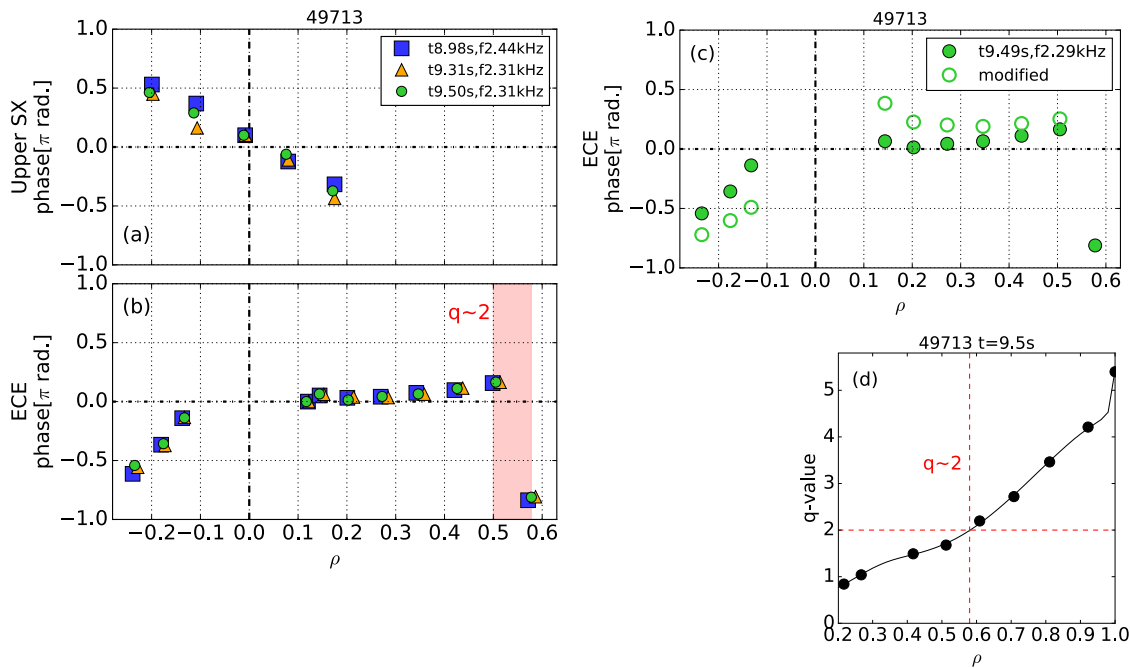


Fig. 4 The radial profile of the phase obtained with cross spectrum of Fourier transform at 9 s, 9.3 s, and 9.5 s from (a) SX measurement and (b) ECE measurement. (c) The radial profile of the phase and the modified phase at 9.5 s. (d) The  $q$ -profile at 9.5 s obtained with MSE measurement and the MEUDAS code. In (a), (b), and (c), the coherence is calculated between the channel viewing near the axis and other channel. The phases where the coherence is high are shown. In (c), the data with the poloidal angle  $\sim 0.5 \pi$  rad. is not shown because the error of the modification near the axis can be large as explained in Appendix A. Here, the relative amplitudes of the fluctuations are  $\delta V_{sx}/V_{sx} \sim 0.017$  at the core,  $\delta T_e/T_e \sim 0.0023$  around  $\rho = 0.12$ , and  $\delta T_e/T_e \sim 0.019$  around  $\rho = 0.51$  ( $q \sim 2$ ).  $V_{sx}$  is the amplitude of the SX signal and  $\delta V_{sx}$  is the fluctuation component of the SX signal due to the  $m/n = 2/1$  NTM.  $T_e$  is the electron temperature and  $\delta T_e$  is the fluctuation component of the electron temperature due to the  $m/n = 2/1$  NTM. In (d), the black circles are the estimated safety factor of MSE channels. Because the measure points do not exist in the core region  $\rho < 0.215$ , the  $q$ -profile is not shown in the core region.

are observed in SX signals and the electron temperature in ECE measurement with the mode frequency of the  $m/n = 2/1$  NTM. Because the observed mode frequencies are the same, the phase profile can be evaluated in both measure-

ments. The phase profiles are calculated at three times in the discharge of Fig. 3. In the time window of Fig. 3,  $\beta_p$  (an indicator of the saturated size of the magnetic island of NTMs [31]), the total injection power of NBIs, and the

radial component of the magnetic fluctuation due to the  $m/n = 2/1$  NTM are nearly constant, indicating that the plasma is the steady state and the size of the magnetic island is almost unchanged. As shown in Fig. 4 (a), the unchanged phase profiles of  $m = 1$  structure in SX signals are observed, indicating the rotating  $m = 1$  structure. Note that, in this study, the  $m = 1$  structure is not measured with magnetic probes arranged poloidally. This might be due to the small amplitude of the  $m = 1$  component compared with the  $m = 2$  component. In the case of ECE measurement, the change of the phase is just  $0.14\pi$  rad. across  $\rho = 0$  and is significant in the high field side where the sign of  $\rho$  is minus. Here,  $\rho$  is the volume-averaged minor radius. This is because the difference of the poloidal angles of the measured points across  $\rho = 0$  is not  $\pi$  rad. on the horizontal plane for ECE measurement as seen in Fig. 2. In Fig. 4 (c), the estimated phases projected on the midplane are shown assuming the measured points of ECE measurement are moved to the midplane keeping the measured minor radius. As seen in the estimated phase profile of Fig. 4 (c), the difference of the phase across  $\rho = 0$  is about  $\pi$  rad., indicating  $m = 1$  structure. The detailed description of the modification is found in Appendix A. The  $m = 1$  structure in SX measurement and ECE measurement seems to have  $n = 1$  structure because the relationship between the phase profiles around  $\rho = 0$  in SX measurement and ECE measurement (Figs. 4 (a) and (b)) can be reproduced in synthetic images assuming the existence of the  $m/n = 1/1$  mode structure in the core, which is the same method as a method in the previous study [20]. The phase difference  $\pi$  around the  $q = 2$  surface of Fig. 4 (d) in ECE signals is due to the  $m/n = 2/1$  NTM. Therefore, the unchanged phase profiles of  $m/n = 1/1$  component and  $m/n = 2/1$  component are obtained, indicating phase locking. The observed phase locking is corresponding to the characteristics (1) and (2) of the coupling because the observed phase profiles can be reproduced in synthetic images assuming the coupling. Note that the same phase from  $\rho = 0.1$  to  $\rho = 0.5$  in ECE measurement is considered due to the  $m = 1$  “helical deformation” mentioned in the previous study [20].

Because the observed mode frequency with the coupling can be several kHz in this study as well as  $\sim 20$  Hz in the previous study [20], the resistive wall does not induce the coupling. In addition, the coupling is observed when the mode frequency of an  $m/n = 2/1$  NTM is 6 kHz. This indicates that the observed HCs are not induced by the plasma control system of JT-60U. Because the plasma control system controls the position of the plasma with 4 kHz by changing the coil current, the fluctuation having the frequency with 6 kHz cannot be induced.

As discussed on Fig. 4, the  $n = 1$  HC and the  $m/n = 2/1$  NTM have the same mode frequency. If the observed  $n = 1$  HC rotates with the bulk plasma like NTMs, the relationship,  $f_{\text{mode}} \sim n \times f_{\text{tor}} - m \times f_{\text{pol}}$ , should be satisfied [33]. Here,  $f_{\text{mode}}$  is the “observed” mode fre-

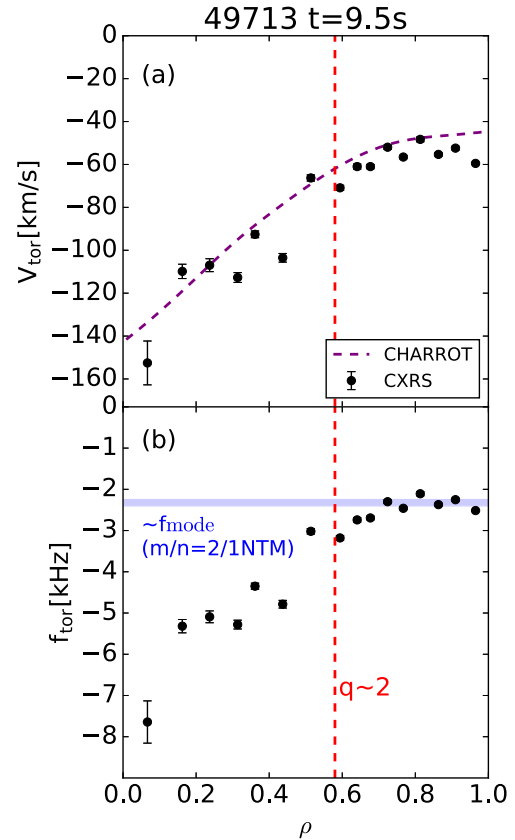


Fig. 5 The radial profile of the toroidal velocity ( $V_{\text{tor}}$ ) and the toroidal rotation frequency ( $f_{\text{tor}} = V_{\text{tor}}/2\pi R_m$ ) measured by CXRS measurement at 9.5 s. Here,  $R_m$  is the major radius where  $V_{\text{tor}}$  is measured. In (a), the purple dashed line is the toroidal velocity of bulk deuterium plasma estimated by the CHARROT code [34, 35]. In (b),  $f_{\text{mode}}$  is the observed mode frequency of the  $m/n = 2/1$  NTM. Here, the monotonic increase in the toroidal velocity profile is due to the net toroidal torque to the ctr-direction from the tangential NBI.

quency,  $f_{\text{tor}}$  is the toroidal rotation frequency defined by  $V_{\text{tor}}/(2\pi R_m)$ , and  $f_{\text{pol}}$  is the poloidal rotation frequency defined by  $V_{\text{pol}}/(2\pi \rho_m)$ . Here,  $V_{\text{tor}}$  ( $V_{\text{pol}}$ ),  $R_m$ , and  $\rho_m$  are the toroidal (poloidal) rotation velocity, the major radius where  $V_{\text{tor}}$  is measured, and the minor radius where  $V_{\text{pol}}$  is measured, respectively. In this study,  $f_{\text{pol}}$  is ignored because the net toroidal torque induced by the tangential NBIs are applied and  $V_{\text{tor}}$  is sufficiently larger than  $V_{\text{pol}}$  estimated with the CHARROT code [34, 35] assuming the effective ion charge  $Z_{\text{eff}}$  is 2. The absolute values of the poloidal flow of the bulk deuterium plasma are smaller than 0.6 km/s from core region to the edge region. Figure 5 shows the radial profile of (a)  $V_{\text{tor}}$  measured by CXRS measurement and (b)  $f_{\text{tor}}$  at 9.5 s of 49713 in Fig. 3, assuming that the rotation velocity of the bulk deuterium plasma and the carbon plasma as impurity are the same. The purple dashed line in Fig. 5 (a) is the calculated toroidal rotation velocity profile of the deuterium by the CHARROT code. The profile of the toroidal rotation velocity by CXRS

measurement and the profile by the CHARROT code are consistent, indicating that it is valid to use  $V_{\text{tor}}$  of carbon plasma as the rotation velocity of the bulk deuterium plasma. The mode frequency of the  $m/n = 2/1$  NTM in Fig. 3 (c) is about 2.3 kHz and is consistent with  $f_{\text{tor}}$  around the  $q = 2$  surface as seen in Fig. 5 (b). On the other hand, the observed frequency of the  $n = 1$  HC is the same as that of the  $m/n = 2/1$  NTM and is different from  $f_{\text{tor}} \sim 5$  kHz around the  $q = 1$  surface ( $\rho \sim 0.2$ ) in Fig. 5 (b) (see also  $q$ -profile in Fig. 4 (d)). If the observed  $n = 1$  HC is the resonant mode localized around the  $q = 1$  surface, the observed frequency should be consistent with  $f_{\text{tor}}$  around the  $q = 1$  surface. The coincidence between the observed frequency of the  $m/n = 1/1$  mode and  $f_{\text{tor}}$  is usually observed with the resonant  $m/n = 1/1$  internal kink resulting in the sawtooth collapse [36]. The difference between the observed frequency of the  $n = 1$  HC and  $f_{\text{tor}}$  around the  $q = 1$  surface in Fig. 5 indicates that the possibility of the excitation of MHD modes except for the resonant  $m/n = 1/1$  internal kink localized at the  $q = 1$  surface. Here, in this study, we investigate the possibility that the observed  $n = 1$  HC is a non-resonant mode.

### 3.2 $n = 1$ HC with $m/n = 2/1$ NTM having $q_{\text{min.}} > 1$

In Sec. 3.1, we indicate the possibility that the observed  $n = 1$  HC is a non-resonant mode. However, in Fig. 4 (d), the  $q$ -profile is the normal shear where  $q_{\text{min.}}$  is nearly unity. Therefore, we have to examine whether the  $m/n = 1/1$  structure is observed with  $q_{\text{min.}} \gg 1$  or not. This is examined at 7.55 s in 48885 where an  $m/n = 2/1$  NTM is observed. The plasma discharge 48885 is for a high  $\beta$  plasma experiment [32] and the weak shear profile is made for forming the internal transport barrier (ITB). With the weak shear profile,  $q_{\text{min.}}$  tends to greater than unity.

The extended view of 48885 is shown in Figs. 6 (a) - (d). In Fig. 6 (a), the  $n = 1$  mode, whose mode frequency is about 200 Hz, is observed in the magnetic fluctuation before 7.55 s. The mode frequency of the  $m/n = 2/1$  NTM is relatively slow ( $\sim$  Hz) after the decrease of the mode frequency at 7.55 s, which is the same object as the LFM described in the previous study [20]. And in Figs. 6 (b) - (d), the coherent fluctuations are observed in SX signals and the electron temperature with the mode frequency of the  $n = 1$  mode. In Fig. 6 (b), because the  $q = 2$  surface locates around  $\rho = 0.6$  and the phases of the fluctuations

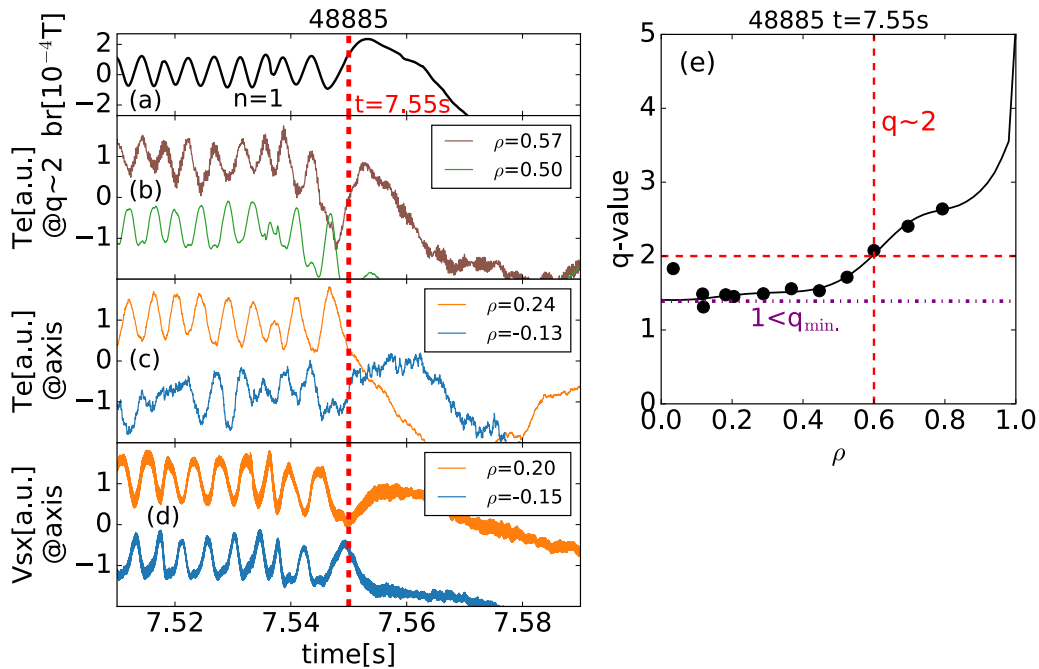


Fig. 6 The time evolution of (a) the magnetic fluctuation, (b) the electron temperature at  $\rho = 0.50$  and  $0.57$  around the  $q = 2$  surface, (c) the electron temperature at  $\rho = -0.13$  and  $0.24$ , and (d) SX signals from  $\rho = -0.15$  and  $0.2$ . (e) The  $q$ -profile at 7.55 s estimated with MSE measurement and the MEUDAS code. In (e), the black circles are the estimated safety factor of MSE channels. At 7.55 s, the plasma parameters are the plasma current  $\sim 0.9$  MA, the magnitude of magnetic field at magnetic axis  $\sim 1.52$  T, the major radius  $\sim 3.43$  m, the minor radius  $\sim 0.92$  m, the radial position of axis  $\sim 3.53$  m, the vertical position of axis  $\sim 0.2$  m, the plasma elongation  $\sim 1.39$ , the plasma triangularity  $\sim 0.38$ , the plasma volume  $\sim 72$  m<sup>3</sup>, and  $q_{95} \sim 3.5$ . This discharge is made after the installation of ferrite steel tiles on the vacuum vessel to reduce the toroidal magnetic field ripple [30]. Here, the difference in the poloidal angle between the measured points across  $\rho \sim 0$  in ECE measurement are about  $\pi$  rad. unlike the case in Fig. 4 (b) because the vertical position of axis and the measured plane by ECE measurement are on the same horizontal plane.

are inverted between  $\rho \sim 0.50$  and  $\rho \sim 0.57$ , it is confirmed that the observed  $n = 1$  mode is an  $m/n = 2/1$  NTM. As well as the phases around the  $q = 2$  surface, the phases are inverted between signals across  $\rho = 0$  in Figs. 6(c) and (d), indicating the  $m = 1$  structure. The  $m = 1$  structure in SX measurement and ECE measurement seems to have  $n = 1$  structure because the relationship between the phase profiles across  $\rho = 0$  in SX measurement and ECE measurement (Figs. 6(c) and (d)) can be reproduced in synthetic images assuming the existence of the  $m/n = 1/1$  mode structure in the core which is the same as the result in the previous subsection. As seen in Figs. 6(a) - (d), the phases are locked (phase locking). Because the observed phases can be reproduced in synthetic images assuming the coupling, the characteristics (1) and (2) of the coupling are confirmed in this discharge as well.

The  $q$ -profile estimated with the MEUDAS code is shown in Fig. 6(e). In this estimation, the polarization angle is averaged from 7.54 s to 7.56 s for the reconstruction of the MHD equilibria to average the non-axisymmetric effect of the mode structure. As shown in Fig. 6(e),  $q_{\min.}$  at 7.55 s is about  $1.4 > 1$ . Even if MHD equilibria considering the error of the measured polarization angle in the inner side of  $\rho = 0.21$  are evaluated, the lower limit of  $q_{\min.}$  is evaluated as 1.12, indicating that  $q_{\min.} > 1$ . These evaluations indicate that the observed  $m/n = 1/1$  mode is a non-resonant mode.

#### 4. Causality between $n = 1$ HC and $m/n = 2/1$ NTM

In the previous section, it is found that  $n = 1$  non-resonant HCs and  $m/n = 2/1$  NTMs are coupled. In this section, the causality between  $n = 1$  HCs and  $m/n = 2/1$  NTMs is investigated to consider the excitation mechanism of the  $n = 1$  HC and the coupling.

As shown in Fig. 6(e), the  $n = 1$  HC is observed with the low magnetic shear. In a plasma with the low magnetic shear, it was reported that the  $m/n = 1/1$  infernal mode can induce the  $m/n = 2/1$  NTM [37, 38]. However, in this study, it is not observed that an  $n = 1$  HC firstly appears and an  $m/n = 2/1$  NTM subsequently appears, excluding the case with the sawtooth collapse which occasionally induces NTMs [39]. This fact suggests the possibility that the  $m/n = 2/1$  NTM induces the  $n = 1$  HC.

Here, the fluctuation in the core and the fluctuation around the  $q = 2$  surface are investigated when an  $m/n = 2/1$  NTM is stabilized by ECCD around the  $q = 2$  surface in 49573 as shown in Fig. 7, which is one of the series of discharges in Table 1. As seen in Fig. 7, the magnetic fluctuation and the temperature fluctuation by the  $m/n = 2/1$  NTM on the  $q = 2$  surface (blue) is stabilized by ECCD around the  $q = 2$  surface until 11 s as indicated by the vertical dashed red line. As seen in Fig. 7(b), the temperature fluctuation by the  $n = 1$  HC from core (red) also disappears until the vertical red line. Because the temperature

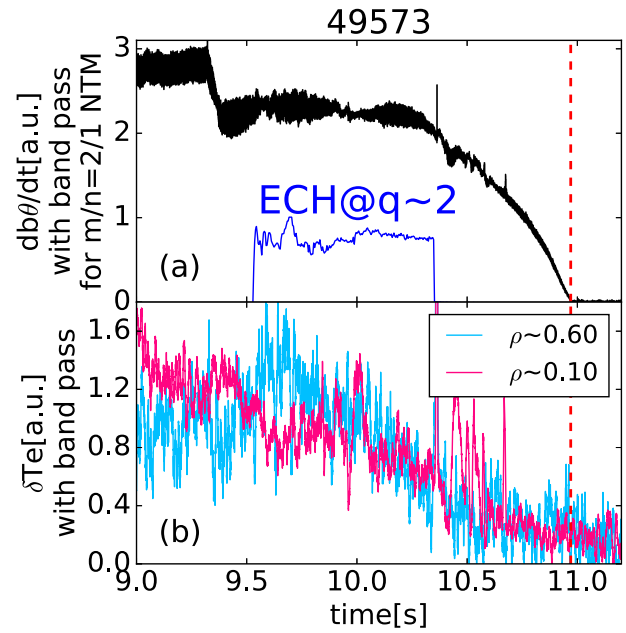


Fig. 7 The time evolution of (a) the magnetic fluctuation and (b) amplitudes of signals from ECE measurement with the band pass filter to obtain the component due to an  $m/n = 2/1$  NTM. In (a), the blue line indicates the pulse pattern of ECH (ECCD) around  $q = 2$  surface to stabilize the  $m/n = 2/1$  NTM. The  $m/n = 2/1$  NTM is stabilized around 11 s according to the magnetic fluctuation. In (b), the signals measured at  $\rho = 0.108$  and  $0.602$  are shown.

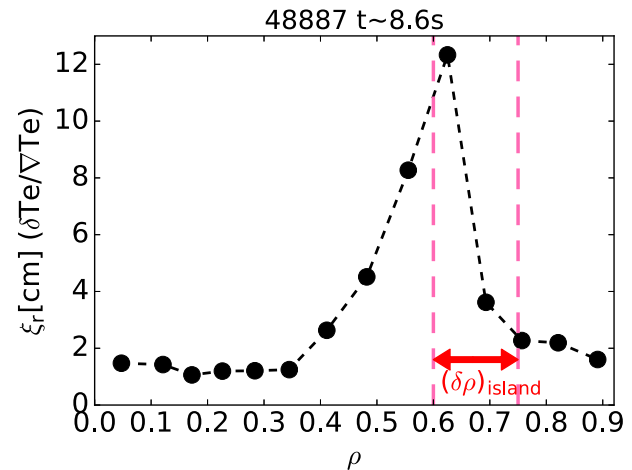


Fig. 8 The profile of the radial displacement  $\xi_r$  estimated with ECE measurement around 8.6 s in 48887 when an  $m/n = 2/1$  NTM and an  $n = 1$  HC is observed.  $\xi_r$  is estimated as  $\delta Te / \nabla Te$ .  $\delta Te$  and  $\nabla Te$  are the fluctuation component of the electron temperature due to the  $m/n = 2/1$  NTM and the radial derivative of the electron temperature profile, respectively. Here, the mode frequency of the  $m/n = 2/1$  NTM is about 4 Hz. The MHD equilibrium is calculated at 8.66 s. The region where the flattening of the ion temperature profile obtained with CXRS measurement is shown by the red arrow. The width of the  $m/n = 2/1$  magnetic island is about  $(\delta \rho)_{\text{island}} \sim 0.15$ .

fluctuation in the core region due to the  $n = 1$  HC disappears with the stabilization of the  $m/n = 2/1$  NTM at the  $q = 2$  surface, it is natural to consider that the  $n = 1$  HC is induced by the  $m/n = 2/1$  NTM.

This perspective,  $n = 1$  HCs are driven by  $m/n = 2/1$  NTMs, is supported by observation of the radial displacement  $\xi_r$  with the coupling of an  $n = 1$  HC and an  $m/n = 2/1$  NTM in Fig. 8. As shown in Fig. 8,  $\xi_r$  is about  $1 \sim 1.5$  cm in the core region, which seems to be the  $m/n = 1/1$  component of  $\xi_r$ ,  $(\xi_r)_{1,1}$ . On the other hand, the  $m/n = 2/1$  component of  $\xi_r$  around the  $q = 2$  surface,  $(\xi_r)_{2,1}$ , is much larger than  $(\xi_r)_{1,1}$  in the core region. The larger  $(\xi_r)_{2,1}$  indicates that the driving source of the  $n = 1$  HC is the  $m/n = 2/1$  NTM.

## 5. Discussion

As discussed in Sec. 3.1, the same observed frequencies of  $m/n = 2/1$  NTMs and  $n = 1$  HCs cannot be explained by rigid plasma rotation. Here, if the  $n = 1$  HC is the sideband of the  $m/n = 2/1$  NTM, the same observed frequency and the characteristic (1) of the coupling can be explained. However, the way that the  $m/n = 2/1$  NTM drives the spatially separated  $n = 1$  HC in the core region is non-trivial. Here we propose a classical forced magnetic reconnection (FMR) model [40,41] based on the quasi-linear MHD model to explain the non-resonant mode and the characteristic of the coupling (1). As discussed in Sec. 4 on the causality, it appears that the  $m/n = 2/1$  NTM is induced firstly and the  $n = 1$  HC is induced subsequently. If the  $m/n = 2/1$  NTM exists, the  $m/n = 2/1$  current filament can induce the  $m/n = 1/1$  mode as the sideband at the “ $q = 2$ ” rational surface, not at the “core”. Once the  $m/n = 1/1$  mode is excited at the “ $q = 2$ ” surface, the  $m/n = 1/1$  mode propagates into the core with Alfvénic speed [40] until causing the discontinuity at the rational surface, or surviving as an Alfvén eigen modes (non-resonant mode). The former discontinuity causes the FMR and also forms the  $m/n = 1/1$  mode at the core. Thus, from the FMR point of view, the  $m/n = 2/1$  NTM can drive the spatially separated  $n = 1$  HC regardless of the resonant or the non-resonant mode. The schematic view of the excitation mechanism is shown in Fig. 9. Regarding the phase relation, metrics are all even under 1st order large aspect ratio approximation. Therefore, the phase of the  $n = 1$  mode at the  $q = 2$  surface is the same with that of the  $m/n = 2/1$  NTM. The  $m/n = 1/1$  mode at the  $q = 2$  surface is the boundary condition for the  $m/n = 1/1$  mode at the core. Thus, the  $m/n = 1$  mode is in-phase with the  $m/n = 2/1$  NTM, which is not consistent with the observation, while it successfully explains the two modes have the same observed frequency and the characteristic (1) of the coupling,  $f_{m/n=1/1(\text{HC})} = 2f_{m/n=2/1(\text{TM})}$ . The discrepancy may be explained by the theoretical explanation of the phase locking with the dynamical stability in [42] and its experimental validation in [43]. The further study is

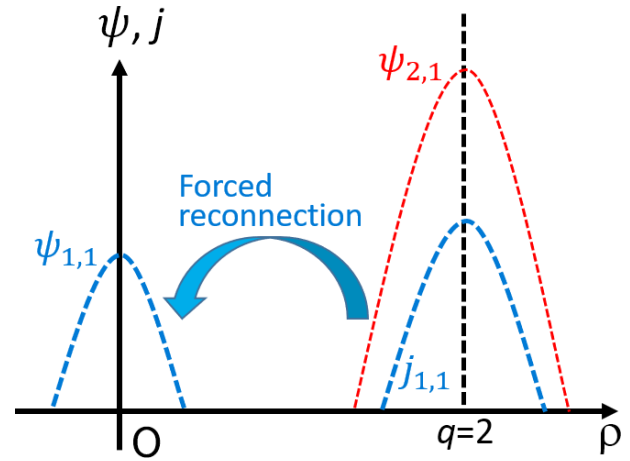


Fig. 9 Schematic view of the excitation mechanism of the  $n = 1$  HC by the  $m/n = 2/1$  NTM.  $\psi_{2,1}$  and  $\psi_{1,1}$  are  $m/n = 2/1$  and  $m/n = 1/1$  component of the poloidal flux, respectively.  $j_{1,1}$  is  $m/n = 1/1$  component of the plasma current as the sideband.

required on this issue.

Though the revisited quasi-linear MHD model can explain the non-resonant characteristic and the characteristic (1) of the coupling, there are some issues to deal with in future works. For example, the revisited model is just for the linear phase and the observation of the nonlinear phase, such as the radial displacement in Fig. 8, cannot be treated. In addition, the characteristic (2) of the coupling and “helical deformation” mentioned in [20] should be also explained. The nonlinear simulations are required to investigate these issues.

In our previous study [20], the similarity between quasi-stationary modes (QSMs) in the JET tokamak [44, 45] and our observations is discussed. QSMs are the low frequency modes ( $\sim 10$  Hz) and showing the  $m = 1$  structure with  $m/n = 2/1$  modes, which is the same as  $n = 1$  HCs with  $m/n = 2/1$  NTMs in our study. Though QSMs were only observed with  $\sim 10$  Hz, we show the coupling with the higher observed frequency with several kHz and introduce the possibility of the non-resonant  $m = 1$  mode in the core. In consideration of the similarity between QSMs and  $n = 1$  HCs in our study, there is possibility that QSMs are accompanied by the non-resonant mode and this perspective should be examined.

## 6. Summary and Conclusion Remarks

In summary, in this paper, we have investigated the excitation mechanism of  $n = 1$  HCs coupled with  $m/n = 2/1$  NTMs described in the previous study [20]. It is found that the coupling is observed with the mode frequency from several Hz to 6 kHz. This indicates that the resistive wall and the plasma control system do not induce the coupling.

In addition,  $n = 1$  HCs are found to be the non-resonant mode from two observations. First, it is observed that  $n = 1$  HCs do not rotate with the plasma around the  $q = 1$  surface in the core region by means of the comparison of the plasma rotation frequency  $f_{\text{tor}}$  and the observed mode frequency  $f_{\text{mode}}$ . Second, the coupling is observed even when  $q_{\text{min.}} > 1$ . Then the causality between  $n = 1$  HCs and  $m/n = 2/1$  NTMs is investigated with the observation of the disappearance of the electron temperature fluctuation in the core with the stabilization of an  $m/n = 2/1$  NTM, indicating that  $n = 1$  HCs are induced by  $m/n = 2/1$  NTMs. We revisit a traditional FMR based on the quasi-linear MHD model where the  $n = 1$  HC is induced directly by the sideband of the current for the  $m/n = 2/1$  NTM. The model can explain the non-resonant characteristic and the characteristic of the coupling,  $f_{m/n=1/1(\text{HC})} = 2f_{m/n=2/1(\text{TM})}$ . Nonlinear MHD simulations are still required to explain the characteristics in the nonlinear phase observed at experiments in future.

In this study, we report observations of non-resonant  $n = 1$  HCs coupled with  $m/n = 2/1$  NTMs for the first time and revisit the traditional quasi-linear MHD model where the  $m/n = 1/1$  mode is induced as the sideband of  $m/n = 2/1$  mode to explain the observed characteristics. In other tokamaks [4] or an RFP [46], it was observed that the external magnetic perturbations induced the helical structure in the plasma. If the  $n = 1$  HC is surely induced by the  $m/n = 2/1$  NTM as proposed in this study, it can be considered that the  $m/n = 2/1$  NTM plays the role of the external magnetic perturbations to induce the helical structure with the same mechanism in other toroidal devices. This also should be examined theoretically and experimentally.

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## Appendix A. Method to Modify the Phase in Sec. 3.1

If we distinguish the poloidal number in the core, the change of the poloidal angle of the measured points should be around  $\pi$  rad. across  $\rho = 0$ . However, as shown in Fig. A (a), the poloidal angles of ECE measurement in the core region changes  $0.7\pi$  rad. Therefore, it is required to modify the observed phase considering these poloidal angles to investigate the mode structure. Here, as shown in Fig. A (b), if the measured point before the movement is located as the lower field side from the magnetic axis, the measured point is moved to the point on the midplane in the lower field side keeping the minor radius. In the opposite case, the measured position is moved to the higher field side. This method to modify the phase is valid when the single mode having a certain poloidal number covers the region for the estimation. This method is also affected by the accuracy of the estimation of the MHD equilibrium.

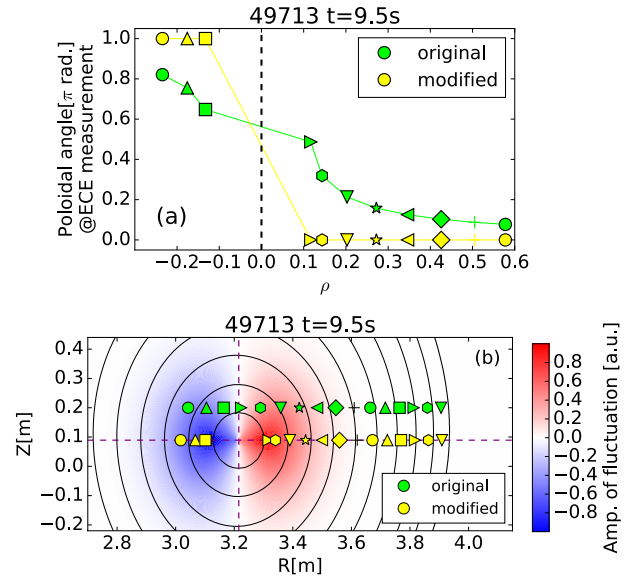


Fig. A (a) The radial profile of the poloidal angle measured by ECE measurement. (b) The flux surfaces in a poloidal cross section and the measured positions of ECE measurement. In (b), the contour map shows the amplitude of the fluctuation of an  $m = 1$  mode in the core region. In figure (a) and (b), the green and yellow markers mean the original measured poloidal angles (points) by ECE measurement and the modified measured poloidal angles (points) with the method explained in Appendix A, respectively.

The error due to the estimation of the MHD equilibrium is more sensitive near the estimated axis.

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