

高パワー加熱を用いた LHD 長時間放電における不純物挙動 Impurity behavior in LHD long pulse discharges with high power heating

中村幸男, 田中謙治, 吉村信次, 吉沼幹朗, 鈴木千尋, 田村直樹, B. J. Peterson
NAKAMURA Yukio, TANAKA Kenji, YOSHIMURA Shinji, YOSHINUMA Mikiro, SUZUKI Chihiro,
TAMURA Naoki, Byron Jay PETERSON

核融合研
NIFS

Impurity behavior has been mainly investigated so far in long pulse discharges with low NBI heating power (< 3 MW). The upgrade of NBI heating systems ($P_{\text{nbi}} \sim 10$ MW, $t_d \sim 5$ s) enables us to maintain high performance plasmas for long time. Now we can study impurity behavior in long pulse discharges with high temperature plasma. Here, we report the experimental results in long pulse discharges with high NBI heating power ($P_{\text{nbi}} = 3 \sim 10$ MW).

Figure 1 shows a typical long pulse discharge with high NBI heating power. In the early stage of this discharge, the total heating power is about 10 MW and the heating power steps down to 6 MW at around 6 s. The line averaged radiation intensity (AXUVD signal) and the intrinsic impurity line intensity (Fe XXIII) at the central chord of plasma increase gradually and seem to be saturated. The step down of heating power brings out a discontinuous rise of the radiation and the impurity line intensity. The measurements of local radiation power with bolometer arrays show a hollow profile in the high power phase and a peaked profile in the low power phase, as shown in Fig. 2, where the normalized radiation profiles are plotted for various heating powers. The peaked radiation profile also appears in a quasi-steady state discharge with 6 MW. As found in the previous

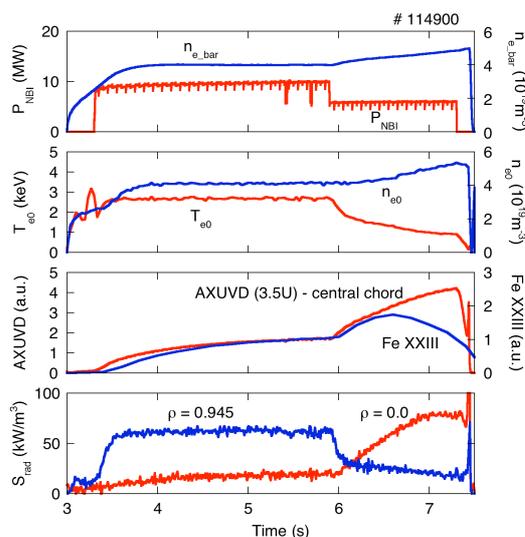


Fig. 1. NBI long pulse discharge with different heating powers of 6 MW and 10 MW

study [1,2], there is an impurity accumulation window in the operational regime of hydrogen discharges on LHD and the plasma parameters in these discharges corresponds to those in the accumulation window. However, no peaked radiation profile is observed in the high power discharge with 10 MW. Another important feature is to enhance the radiation in the edge plasma region.

According to neoclassical theory of impurity transport in helical plasma, the convection term depends on radial electric field (positive: outward, negative: inward) and normalized temperature gradient (outward). In these discharges, the radial electric fields are negative and almost the same. For hydrogen discharges with various heating powers (3~10 MW), we measured radiation profiles in the range of accumulation window. Figure 3 shows the dependence of radiation peaking factor on the normalized electron temperature gradient. Although the temperature gradient depends on the heating power, the normalized temperature gradient does not change largely. However, no peaking of radiation is observed in the discharges with high heating power (10 MW). Such a drastic change suggests that there is a new type of outward convection (due to plasma turbulence etc.) in the core region for high power heating. This may be related to impurity hole observed in high T_i mode in LHD. Further investigations on carbon density profile and density fluctuation will be discussed.

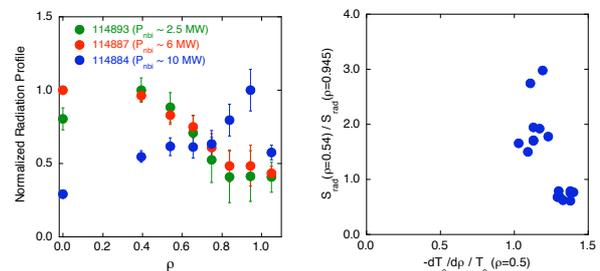


Fig. 2. Radiation profiles for various heating powers

Fig. 3. Dependence of radiation peaking factor on normalized T_e gradient

References

- [1] Y. Nakamura et al., PPCF 44 (2002) p.2121
- [2] Y. Nakamura et al., NF 43 (2003) p.219