## **Central Deuteron Temperature Derived from Total Neutron Emission Rate in Electron Cyclotron Heated LHD Plasmas**

Kunihiro OGAWA<sup>1,2)</sup>, Mitsutaka ISOBE<sup>1,2)</sup>, Takeo NISHITANI<sup>1)</sup>, Hiroki KAWASE<sup>2)</sup>, Neng PU<sup>2)</sup> and LHD Experiment Group

<sup>1</sup>National Institute for Fusion Science, National Institutes of Natural Sciences, Toki-city, Gifu 509-5292, Japan <sup>2</sup>SOKENDAI, Toki-city, Gifu 509-5292, Japan

(Received 6 June 2017 / Accepted 2 August 2017)

Total neutron emission rate from Large Helical Device plasmas is measured by using a neutron flux monitor newly installed for deuterium operations. Deuteron temperature is derived from the total neutron emission rates in electron cyclotron heated Large Helical Device plasmas. Time evolution of central deuteron temperature obtained by the neutron flux monitor approximately agrees with central impurity ion temperature measured by the EUV spectrometer.

© 2017 The Japan Society of Plasma Science and Nuclear Fusion Research

Keywords: neutron diagnostics, total neutron emission rate, deuteron temperature, LHD

DOI: 10.1585/pfr.12.1202036

Deuterium operation on Large Helical Device (LHD) was initiated in March 2017 in order to achieve higher temperature and higher density plasmas [1]. In deuterium experiments, fusion reaction products such as neutrons become newly available as a signal to diagnose compared with hydrogen experiment. Total neutron emission rate  $(S_n)$  measurement on thermal plasmas has been used to determine the central deuteron temperature  $(T_{d0})$  in tokamaks and middle sized stellarators/helical devices [2-6]. Recently, a neutron flux monitor (NFM) composed of three fission chambers, a <sup>10</sup>B counter, and two <sup>3</sup>He proportional counters was installed [7] and working successfully for measurement of  $S_n$  in LHD. As a result, we are able to obtain the time-resolved deuteron temperature in electron cyclotron (EC) heated deuterium plasmas by means of the NFM.

Figure 1 shows the typical time evolution of the deuterium plasma discharge with EC heating. Total EC heating power ( $P_{\text{ECH}_{total}}$ ) was reduced in two steps from 3 MW to 1 MW and line-averaged electron density ( $n_{e_avg}$ ) was gradually increased to 2 × 10<sup>19</sup> m<sup>-3</sup> throughout the discharge. Deuterium ratio {D/(H + D)} evaluated by H $\alpha$  and D $\alpha$  diagnostics was around 0.9. Maximum total neutron emission rate ( $S_n$ ) and total neutron yield measured by the NFM are 0.7 × 10<sup>11</sup> neutrons per second, and 1.0 × 10<sup>11</sup> neutrons per shot, respectively. Radiation power ( $P_{rad}$ ) and the central electron temperature ( $T_{e0}$ ) were measured by a resistive bolometer and a Thomson scattering diagnostics (TSD), respectively.

In EC heated plasmas,  $S_n$  can be expressed by

$$S_{\rm n} = \frac{1}{2} \int_{r=0}^{a} n_{\rm D}(r)^2 \langle \sigma v \rangle_{\rm DD-n}(T_{\rm d}(r)) \mathrm{d}V(r). \tag{1}$$

ECH\_tota [MW] 0.8 [a.u.] Ha D 0 D/(H+D) 0.0 n<sub>e\_avg</sub> [10<sup>19</sup>m<sup>-3</sup>] 0.8 S<sub>n</sub> [10<sup>11</sup> n/s]<sup>0.4</sup> 0.0 400 P<sub>rad</sub> [kW] 200 Teo [keV] 5 Time [s]

#137117 Bt = 2.75 T, Rax =3.60 m

Fig. 1 Typical time evolution of EC heated plasma.

author's e-mail: kogawa@nifs.ac.jp

Here r, a,  $n_D$ ,  $\langle \sigma v \rangle_{DD-n}$ ,  $T_d$ , and V are minor radius, averaged plasma minor radius, deuteron density, reaction rate of the  $D(d, n)^3$ He reaction when deuterons form the Maxwellian velocity distribution, deuteron temperature, and the volume of each flux surface, respectively. In this ion temperature derivation,  $S_n$  is measured by <sup>10</sup>B counter with the conversion factor derived by in-situ neutron calibration. The profile of  $n_{\rm D}$  is deduced from  $n_{\rm e}$  profile with assuming the  $Z_{eff}$  range from 1.5 to 2.5 and main impurity of C<sup>6+</sup> [8], and D/(H+D) obtained by D $\alpha$  and H $\alpha$  measurement.  $D(d, n)^{3}$ He reaction rate is given from the analytic model [9]. Deuteron temperature profile is assumed to be in the parabolic shape,  $T_d(r) = T_{d0} \times \{1 - (r/a)^2\}$ . The intervals of  $T_{d0}$  in this calculation is 0.05 keV. Volume of each surface is given from the equilibrium reconstructed using VMEC2000 [10].

Figures 2 (a) and (b) show the profiles of electron temperature  $(T_e)$  and  $n_e$  measured by TSD, and estimated ion density profile at *t* of 5.01 s. We calculated

$$\varDelta = \left| S_{n} - \frac{1}{2} \int_{r=0}^{a} n_{D}(r)^{2} \langle \sigma v \rangle_{DD-n}(T_{d}(r)) dV(r) \right|^{2}$$

for each  $T_{d0}$  to find appropriate  $T_{d0}$  to minimize  $\Delta$  in each time bin. Here, the calculated neutron emission density rate profile at  $T_{d0}$  of 1.40 keV, at *t* of 5.01 s is shown in Fig. 2 (c). Figure 3 shows the time evolutions of  $T_{d0}$  and



Fig. 2 Radial profiles of a)  $T_{e}$ , b)  $n_{e}$  and  $n_{D}$ . c) Neutron emission density profile calculated at  $T_{d0}$  of 1.40 keV and  $Z_{eff}$  of 2.



Fig. 3 Time evolutions of  $T_{i0}$  and  $T_{d0}$ .

central impurity ion temperature measured by EUV spectrometers ( $T_{i0}$ ) which is basic diagnostics for measuring ion temperature in EC heated LHD plasmas. The error bar of  $T_{d0}$  is mainly from the error of in-situ neutron calibration of NFM. Time evolution of  $T_{d0}$  approximately agrees with that of  $T_{i0}$ .

In summary, time evolution of deuteron temperature is derived from the total neutron emission rate measured by the neutron flux monitor in electron cyclotron heated deuterium LHD plasmas. Appropriate time evolution of central deuteron temperature is derived by total neutron emission rate measured using the neutron flux monitor.

This work is supported partly by LHD project budgets (ULGG801, ULHH003, and ULHH034).

- Y. Takeiri *et al.*, "Extension of Operational Regime of LHD towards Deuterium Experiment" in Proceedings of 26th IAEA Fusion Energy Conference, Kyoto, Japan, 17-22 October (2016).
- [2] K.M. Young *et al.*, Plasma Phys. Control. Fusion **26**, 11 (1984).
- [3] O.N. Jarvis, Plasma Phys. Control. Fusion 36, 209 (1994).
- [4] C.L. Fiore and R.L. Boivin, Rev. Sci. Instrum. 66, 945 (1995).
- [5] S. Besshou *et al.*, Nucl. Instrum. Methods **A237**, 590 (1985).
- [6] M. Isobe et al., Rev. Sci. Instrum. 68, 532 (1997).
- [7] M. Isobe et al., Rev. Sci. Instrum. 85, 11E114 (2014).
- [8] X. Huang et al., Plasma Fusion Res. 10, 3402036 (2015).
- [9] H.S. Bosch and G.M. Hale, Nucl. Fusion 32, 611 (1992).
- [10] S.P. Hirshman and J.C. Whitson, Phys. Fluids 26, 3353 (1983).