Density Limit Study on the W7-AS Stellarator

GRIGULL Peter*, GIANNONE Louis, STROTH Ulrich, BORRASS Kurt, BRAKEL Rudolf, BURHENN Rainer, ELSNER Albrecht, FIEDLER Stefan, HACKER Herbert, HARTFUSS Hans Jürgen, HERRMANN Albecht, HILDEBRANDT Dieter¹, KUEHNER Georg, SCHNEIDER Ralf, WAGNER Friedrich, WELLER Arthur, ZHANG Xao Dong² and the W7-AS TEAM Max-Planck-Institut für Plasmaphysik, EURATOM Ass., D-85748 Garching, Germany ¹Bereich Plasmadiagnostik, D-10117 Berlin, Germany

²Institute of Plasma Physics, Academia Sinica, Hefei, P. R. China

(Received: 30 September 1997/Accepted: 22 October 1997)

Abstract

Data from currentless NBI discharges in W7-AS strongly indicate that the maximum density for quasi-stationary operation is limited by detachment from limiters. The threshold density at the edge scales with $P_s^{0.5}B^{0.8}$ (with P_s being the net power flow across the LCMS) which is consistent with an edge based analytic estimation presuming constant threshold downstream temperatures.

Keywords:

stellarator, W7-AS, plasma edge, density limit, limiter

1. Introduction

In currentless stellarator discharges, the maximum achievable plasma density is basically limited by an impurity radiation induced thermal instability (density limit, DL) [1,2] which, however, does not lead to MHD instability with subsequent disruption as is typical for tokamak DL discharges [3-5]. The sequence of events leading to thermal instability is governed by the same basic physics as in tokamaks and should exhibit some similarity, at least in limiter machines. This paper continues previous DL studies on W7-AS [1,2] and concentrates on the starting event of the DL sequence (onset condition) which can be described in terms of edge quantities alone, and which practically determines the actual operational window for quasi-stationary limiter discharges. The aim is to give a status report rather than a comprehensive description.

2. Experimental

The analysis was made for net current-free NBI discharges in W7-AS (R = 2 m, a = 0.18 m) at B =

1.25 and 2.5 T and t = 0.34. The configuration was bounded by two tangential graphite limiters at the top and bottom of an elliptical cross section. The absorbed NBI power was varied between 0.35 and 1.2 MW. The power deposited onto the limiters (from limiterintegrated thermocouples) in discharges with flat-top density below the DL was typically about 80 % of the non-radiated power fraction. For the DL study, the density was slowly ramped up (typically for about 0.4 s) until the discharges collapsed. Particle confinement times are estimated to be less than 50 ms.

3. Results and Discussion

3.1 Core parameters, radiation

With the line-averaged density \bar{n}_{e} ramped up, the stored energy slightly increases up to a more or less pronounced maximum and subsequently rolls over, at first along a time scale of some ten milliseconds and then much faster (see example in Fig. 1). This fast decay coincides with a rapid decrease of the density,

©1998 by The Japan Society of Plasma Science and Nuclear Fusion Research

^{*}Corresponding author's e-mail: grigull@ipp-garching.mpg.de

even though the feedback controlled gas valves fully open in this phase. At B = 2.5 T, the total radiated power P_r (from bolometer cameras) is found to increase approximately as $P_r \propto \bar{n}_e^2(Z_{eff}-1)$ (Z_{eff} from bremsstrahlung) up to a critical density \overline{n}_{e}^{crit} . Above this density the increase becomes much steeper indicating radiative instability. Discharges at B = 1.25 T, where Z_{eff} values were not available, show in general a quite similar evolution of P, with time. Abel-inverted radiation profiles (from 12-channel bolometer camera) are typically hollow below \bar{n}_{e}^{crit} , then increase first at the edge, and become strongly peaked during the collapsing phase. Data from bolometer cameras at three different toroidal positions do not indicate significant radiation asymmetries within the last closed magnetic surface (LCMS); MARFE formation was not observed. Whether the phase after having attained \bar{n}_{e}^{crit} leads to a complete collapse following the example in Fig.1, seems to depend primarily on the external gas programme. In contrast to feedback control, with constant or slowly increasing gas feed the discharges could often be further sustained at a degraded energy level or even oscillated between low and high energy contents. Within the low energy phases, n_e , T_e (from Thomson scattering) and P_r radial profiles indicate a radial shrinking of the plasma column.

3.2 Scrape-off layer (SOL) parameters

The upstream densities n_{es} at the LCMS (from Libeam) increase $\propto \bar{n}_e^{1.1\pm0.1}$ at B = 1.25 T and \propto $\bar{n}_{s}^{1.5 \pm 0.1}$ at B = 2.5 T up to a maximum which coincides with the stored energy maximum, and then decrease at higher \bar{n}_{e} (see example in Fig. 2). LCMS upstream temperatures T_{es} are typically above 60 eV up to n_{es}^{max} and then drop rapidly. The high T_{es} values are consistent with the small wetted limiter areas, short power flux decay lengths λ_q (≤ 1 cm), and long field line connection lengths L_c (\approx 30 m) in W7-AS. Downstream strike-point temperatures T_{ed} (from limiter-integrated Langmuir probes) decrease from slightly below T_{es} at low density to a more or less pronounced pedestal of about 20-25 eV at about n_{es}^{max} and then drop further. The downstream ion saturation currents I_{sat} and densities $n_{\rm ed}$ derived from it increase with $n_{\rm es}$ up to a certain density $n_{es}^{det} < n_{es}^{max}$ and then fall typically by factors of about 1.5 to 5. This indicates reductions of the thermal plasma pressure along field lines by more than a factor of eight and detachment from limiters. Detached phases with high stored energy as shown in Fig.1 do not exhibit $n_{\rm e}$, $T_{\rm e}$ and $P_{\rm r}$ radial profile shrinking. This is in contrast to limiter tokamak DL scenarios



Fig. 1 Typical evolution of core parameters for the lineaverged density \bar{n}_{e} ramped up until collapse. W_{dia} is the stored energy, T_{eo} the electron temperature at the centre; for other symbols see text.

which alternatively show detachment by radial shrinking or MARFE formation. The difference could be due to the radial flux expansion by about a factor of two at the limiter positions in W7-AS.

The consistency of the measured SOL parameters with basic SOL physics is checked by a simple twopoint estimation including the power balance along field lines, momentum balance and the sheath boundary condition [5, 6]:

$$T_{\rm es} = \left(T_{\rm ed}^{7/2} + \frac{7}{4\kappa_{\rm o}} q_{\rm lls}L_{\rm c}\right)^{2/7} \text{ with } q_{\rm lls} = \frac{P_{\rm s}}{\lambda_{\rm q}w} \qquad (1)$$

$$2n_{\rm ed}T_{\rm ed} = (1 - f_{\rm m})n_{\rm es}T_{\rm es}$$
 (2)

$$n_{\rm ed} \sqrt{\frac{2k}{m_{\rm i}}} T_{\rm ed}^{1/2} k(\xi + \gamma_{\rm s} T_{\rm ed}) = (1 - f_{\rm r}) q_{\rm lls}.$$
 (3)

 $P_{\rm s}$, $q_{\rm is}$, w, $f_{\rm r}$, $f_{\rm m}$ and ξ are the net power crossing the LCMS, the upstream parallel power flux density, the wetted limiter width, the SOL energy and momentum loss fractions and the potential energy transferred per electron-ion pair ($\approx 25 \text{ eV}$), respectively. Further symbols have the usual meaning [5].

To visualize the role of momentum losses, the measured P_s and T_{ed} values (beside geometrical parameters and λ_q) and $f_r = f_m = 0$ are used as input in a first step. Then a high recycling scenario with $n_{ed} > n_{es}$ is predicted which clearly was not observed (see the discrepancy between calculated and measured n_{ed} values in Fig. 2). Taking P_s , n_{es} and $I_{sat} = en_{ed}c_sA_p$ (with A_p being the probe area) as input and varying the remaining free parameter f_r for f_m to match the $f_m(T_{ed})$

values for CX momentum losses from Fig. 24 in Ref. [7] results in a detached scenario consistent with the measured data with the exception of T_{ed} . The latter should be < 5 eV near the I_{sat} minimum, see Fig. 3. The example further indicates that both strong momentum and volumetric energy losses from the SOL are required for consistency within this estimate. Though f_r in the SOL could not yet be measured, this explanation seems to be somewhat realistic because the measured $n_{\rm es}$ and $I_{\rm sat}$ values are much less prone to error than the $T_{\rm ed}$ probe values. In the presence of strong $T_{\rm e}$ parallel gradients, the latter are very likely overestimated due to contributions from fast tail electrons from regions further upstream [8]. On the other hand, we cannot exclude that momentum losses due to radial transport [9] contribute to detachment in this geometry. The further analysis is not affected by this uncertainty.

3.3 Operational limit scaling

Detached scenarios could not yet be quasi-stationarily sustained in W7-AS. We therefore focus on the upstream limit density n_{es}^{det} for attached discharges which presently determines the practical operational limit for quasi-stationary limiter discharges. From equations (1)-(3) we get

$$n_{\rm es} = \left(\frac{4\kappa_{\rm o}}{7}\right)^{2/7} \frac{(1-f_{\rm r})}{(1-f_{\rm m})} \frac{q_{\rm lls}^{5/7}}{L_{\rm c}^{2/7}} F(T_{\rm ed});$$

$$F(T_{\rm ed}) = \frac{2T_{\rm ed}^{1/2}}{\sqrt{\frac{2k}{m_{\rm i}}} k(\xi + \gamma_{\rm s} T_{\rm ed})}$$
(4)

Considering carbon (from limiters) as dominating SOL impurity, it can be assumed that detachment starts at a threshold $T_{\rm ed}$ value between about five and ten eV where both radiative and momentum losses from the SOL become efficient. The exact value is not critical because $F(T_{\rm ed})$ is only a weak function of $T_{\rm ed}$ in this range and can be taken as constant. The above described calculation to reproduce the measured $n_{\rm es}$ and $I_{\rm sat}$ values for a detached state yields f_r , $f_m \ll 1$ and $(1 - f_r)/(1 - f_m) \approx$ const. when applied to the attached phases. With this approach we get the onset condition

$$n_{\rm es}^{\rm det} \propto \frac{q_{\rm ls}^{5/7}}{L_{\rm c}^{2/7}} = \frac{P_{\rm s}^{5/7}}{(\lambda_{\rm q}w)^{5/7}L_{\rm c}^{2/7}}$$
 (5)

or, with $(\lambda_q w) \propto P_s^x/(n_{es}^y B^z)$ (see below),

$$n_{\rm es}^{\rm det} \propto \frac{P_{\rm s}^{\alpha} B^{\beta}}{L_{\rm c}^{\delta}}, \ \alpha = \frac{5/7(1-x)}{1-5/7y},$$
$$\beta = \frac{5/7y}{1-5/7y}, \ \delta = \frac{2/7}{1-5/7y} \tag{6}$$



Fig. 2 Temporal evolution of LCMS upstream and downstream parameters. The l_{sat} and H_a line intensity drops at 0.44 s indicate detachment from limiters. A two-point estimation (see text) based on the less reliable probe T_{ed} and neglecting momentum losses predicts high recycling with $n_{ed} > n_{es}$ (lines in the bottom figure) which was not observed.



Fig. 3 SOL momentum and energy loss fractions f_m and f_r , respectively, required to explain the observed reduction of the downstream/upstream plasma pressure at 0.53 s in Fig. 2 by friction with neutrals. The free parameter f_r was varied for f_m to match the $f_m(T_{ed})$ data for CX momentum losses in Ref. [7]. The dotted lines indicate the scatter range of data in Ref. [7].

 $\lambda_{\rm q}$ values derived from SOL $n_{\rm e}$ and $T_{\rm e}$ radial profiles (from fast recipro-cating Langmuir probes) are found to scale as $P_{\rm s}^{0.4\pm0.05} n_{\rm es}^{-0.4\pm0.05} B^{-0.5}$ (with the *B* depend-



Fig. 4 Threshold densities for the onset of detachment from limiters indicated by the $I_{\rm sat}$ drop. Errors are estimated to be about $\pm 10\%$ for the densities and ± 30 kW for the power. $\bar{n}_{\rm e}^{\rm max}$ indicates maximum densities which were transiently achieved during detachment.

ence inferred from profiles at only two different Bvalues, 1.25 and 2.5 T). With tangential limiters, the wetted limiter width slightly increases with λ_{α} , $w \propto$ $\lambda_{\alpha}^{0.4}$. Inserting these values into eq. (6) yields α = 0.53 ± 0.12 , $\beta \approx 0.83$ and $\delta = 0.48 \pm 0.05$ which is consistent with the experimental findings shown in Figs. 4 and 5 (L_c was not varied). Nevertheless, the scaling with B needs further confirmation as high power discharges at low magnetic field in general were less close to quasi- stationarity (deteriorated density control) than those at high field. With respect to the dependence of the limit density on the absorbed NBI heating power we found $n_{\rm es}^{\rm det} \propto P_{\rm h}^{0.6} B^{0.8}$. This, together with the similar scaling with P_s , indicates that the limit should occur at an approximately constant core radiated power fraction which was actually observed ($f_r^{\text{core}} = 0.55 \pm 0.1$). It has to be noted that any long-term effects, for example possible impurity accumulation, are not considered in this study.

4. Summary and Conclusion

Data from currentless NBI discharges in W7-AS show that the maximum achievable density in quasistationary discharges is actually limited by detachment from limiters indicated by a drop of the downstream I_{sat} . Corresponding threshold densities at the edge scale



Fig. 5 Summary of the edge threshold densities in Fig. 4. The scaling pre-factor is valid for $P_{\rm s}$ in MW and B in T.

as $n_{\rm es}^{\rm det} \propto P_{\rm s}^{0.5}B^{0.8}$ which is consistent with an edge based, analytical estimation presuming constant threshold downstream temperatures. Model improvement (B2/EIRENE code) and further experiments with respect to the detached phase and possible long-term effects are planned.

References

- A. Stäbler et al., 14th IAEA Conf. on Plasma Physics and Controlled Nuclear Fusion Res., Würzburg, Germany, 1992, Vol. 2, p. 523.
- [2] L. Giannone et al., Proc. 24th Europ. Conf. on Contr. Fusion and Plasma Physics, Berchtesgaden, Germany, 1997, Vol. IV, p.1565.
- [3] M. Greenwald *et al.*, Nucl. Fusion 28, 2199 (1988).
- [4] J. A. Wesson et al., Nucl. Fusion 29, 641 (1989).
- [5] K. Borrass et al., Nucl. Fusion 33, 63 (1993).
- [6] H. S. Bosch *et al.*, Report IPP 1/281a, Garching, 1994.
- [7] C. S. Pitcher and P. C. Stangeby, Plasma Phys. Control. Fusion 39, 779 (1997).
- [8] J. A. Wesson, private communication.
- [9] G. Herre *et al.*, J. Nucl. Materials **241-243**, 941 (1997).